# Waste Form Strategies for Mo-rich Radioactive Waste

# M.W.A. Stewart, E.R. Vance Australian Nuclear Science and Technology Organisation, ANSTO Lucas Heights, NSW 2234 Australia

#### ABSTRACT

This paper describes a small scoping study examining potential multiphase ceramic waste forms for wastes deriving from U-Mo research reactor fuel reprocessing. These fuels are being developed as replacements for silicide and aluminium fuels. The aim was to identify plausible phases that can be used in combination to achieve waste form monoliths with high waste loadings. These waste streams have unique challenges primarily because they have high Na, P and Mo contents. The approach taken was to utilize the Na and P and Mo to form phases that have been previously studied and are known to be durable. The Mo presents challenges because it is multivalent. In air it exists in the hexavalent state, but it can also be partially reduced to the tetravalent state and will substitute for Ti<sup>4+</sup> in synroc phases such as perovskite, rutile and pyrochlore. Under extremely reducing conditions it will be reduced to the metallic state.

Five compositions were tested. The waste loadings ranged from 40 to ~77 wt%. Powellite (nominally, CaMoO<sub>4</sub>) was one of the main phases formed in all compositions when Ca was added. Powellite was also formed by the coupled substitution of Na and Gd for Ca. Ba and Sr were also incorporated in the powellite. NZP (NaZr<sub>2</sub>P<sub>3</sub>O<sub>12</sub>) and NTP (NaTi<sub>2</sub>P<sub>3</sub>O<sub>12</sub>) were also found as major phases in some of the compositions tested. Attempts to incorporate Na as a Na-Gd-titanate perovskite did not work, and instead the Na tended to react with the Gd and Mo to form powellite. Left over Gd reacted with P to form monazite. Pyrochlore was formed in one sample in which it was a target phase, with Mo in the tetravalent state. This pyrochlore appears to be a Gd-Mo-Ti pyrochlore with Na and some Al incorporated, plus traces of other waste elements. The XRD pattern suggests pyrochlore although the composition as measured suggests that it is a defect pyrochlore with vacancies in the A-site. Ca phosphate phases were also detected in some compositions.

The initial results of this scoping study are promising with the results indicating that waste loadings of ~ 50 wt% or maybe higher are feasible. The composition needs to be refined to eliminate the possibility of forming less durable secondary phases. Powellite is a major phase that forms, however we do not have durability data on this material and some testing would be needed to confirm its durability, particularly the (Ca,Na,Gd)MoO<sub>4</sub> composition.

## INTRODUCTION

There is a shift in research reactor fuel to lower levels of enrichment to minimize proliferation risks<sup>1</sup>. This has resulted in a shift from Al metal clad fuels, which cannot achieve the desired U core densities, to silicide fuels. However, silicide fuels are difficult to reprocess. Hence, Mo fuels are currently being tested and verified as potential replacements for silicide fuels. The use of such fuels will inevitably produce waste streams rich in Mo should they be reprocessed. In addition, there are currently in existence fission product wastes high in Mo, but also high in Na and P<sup>2</sup>. It is therefore desirable to examine potential waste forms that can cope with high Mo waste streams. In this paper we discuss the results of our scoping study into potential waste form strategies for the incorporation of Mo into ceramic waste forms. We have investigated wastes that are not only high in Mo but also P and Na and also contain fission products and process impurities .

The latter waste stream consisted of fission products, some actinides and process chemicals, but was distinguished by its high molar content of Mo, P and Na<sup>3</sup>. Since the work at ANSTO began COGEMA have reported work on incorporating this type of waste in a glass <sup>2,3</sup>. The glass was redesigned from their present R7T7 formulation because of the effect of Mo on the long-term behavior and degradation of the glass. They developed a new formulation and utilized cold crucible technology to make the glass, as the target glasses require processing temperatures of 1200-1300°C. The waste loading targeted by COGEMA was 10 wt% Mo. Given that Mo is ~ 66

wt% of the waste this equates to a waste loading of ~ 15 wt%. The COGEMA glass was opaque and contained "microbeads" that incorporated Mo, Ca, P and Zn. Powellite was detected in the beads. The durability was tested by soxhlet experiments at 100°C and initial alteration rate values of 3 g.m<sup>-2</sup>.d<sup>-1</sup> were obtained. PCT tests in water at 90°C were also conducted. The maximum leach rate reported was 2.5 x  $10^{-2}$  g.m<sup>-2</sup>.d<sup>-1</sup> for B between 91 and 120 days. No other leach data was given <sup>2,3</sup>.

In this paper we discuss the results of a feasibility study into the potential to utilize multiphase ceramics to incorporate Mo-rich wastes that are also high in Na and P. The phases chosen for evaluation were NZP (NaZr<sub>2</sub>P<sub>3</sub>O<sub>12</sub>), NTP (NaTi<sub>2</sub>P<sub>3</sub>O<sub>12</sub>), monazite, pyrochlore, powellite and Na-rare earth (RE)-titanate perovskite (figure 1).

# POTENTIAL PHASES AND FORMULATION STRATEGIES

Previous attempts at ANSTO to substitute Mo for Ti in pyrochlore-rich ceramics indicate some substitution occurs and this is discussed below. If Ca is present the Mo reacts with it to form powellite, CaMoO<sub>4</sub>, even when sintered in argon. Therefore, a modified synroc formulation should be sufficient for a waste that contains fission products and Mo. The situation in actual Mo-rich radioactive waste streams, which can contain not only fission products, but also process chemicals and impurities, is much more complicated. In this case of the waste stream considered here, where the waste contains large amounts of both Na and P, in addition to Mo<sup>2,3</sup>, alternatives to the standard synroc phases must be considered. Single-phase materials are unlikely to be able to incorporate all the elements present, much less provide chemical flexibility to deal with changes in the waste stream chemistry. We have therefore examined the use of multiphase ceramic systems utilizing phases such as NZP (NaZr<sub>2</sub>P<sub>3</sub>O<sub>12</sub>), NTP (NaTi<sub>2</sub>P<sub>3</sub>O<sub>12</sub>), monazite, pyrochlore, powellite and Na-rare earth (RE)-titanate perovskite to handle such waste streams.

The major elements in the waste stream are Na, Mo and P. Therefore, any strategy needs to have phases that can incorporate these elements. Sodium is a problematic element in many waste streams, its inclusion resulting from caustic soda added for pH control, NaNO<sub>2</sub> for redox control of Pu during reprocessing and from sodium carbonate added as a conditioner in the tributyl phosphate in the PUREX process.

Sodium can be incorporated into synroc-C at levels less than ~ 2.5 wt% Na<sub>2</sub>O, without serious degradation to the durability properties of the material, above this amount there is an increase in the leach rate and in the pH of the leach liquor<sup>4</sup>. Radiation damage tests using Cm-244 suggested that at high levels the Na, in conjunction with other waste ions and impurities, forms an intergranular glass<sup>5</sup>. Sodium also results in an appreciable drop in the temperatures required for hot-pressing and sintering of the titanate synroc-C<sup>6</sup>. Additional methods examined to incorporate Na in synroc or other titanates included the coupled incorporation of Na with trivalent rare earth ions in perovskite and the utilization of the freudenbergite phase (Na<sub>2</sub>(Al,Fe)<sub>2</sub>Ti<sub>6</sub>O<sub>16</sub>), which was shown to be durable <sup>7</sup>.

Phosphorus and Na were major components of a proposed single phase waste form family based on NaZr<sub>2</sub>P<sub>3</sub>O<sub>12</sub> (NZP) <sup>8</sup> and NaTi<sub>2</sub>P<sub>3</sub>O<sub>12</sub> (NTP) <sup>9</sup>. The latter has been reportedly capable of incorporating up to 60 wt% waste loadings<sup>9</sup>. These phases were originally examined as waste forms at Pennsylvania State University in the late 1970s to early 1980s<sup>10</sup>. There has been a recent revived interest in them and the related in MO-ZrO<sub>2</sub>-P<sub>2</sub>O<sub>5</sub> (where M = Mg, Ca, Sr or Ba)<sup>11 12</sup> in Russia<sup>13 14 15 16</sup> and elsewhere <sup>17 18 19</sup> as nuclear waste forms, low thermal expansion ceramics and super-ionic conductors<sup>20</sup>. Results on radiation damage tests indicated that NZP-structure material [in this case (Cs<sub>1+x</sub>Nd<sub>x</sub>Zr<sub>2-x</sub>(PO<sub>4</sub>)<sub>3</sub>] had good radiation damage resistance when doped with <sup>239</sup>Pu, although it became metamict when doped with <sup>238</sup>Pu <sup>21</sup>.

Phosphorus can also be used to form other highly durable phases that have been proposed as hosts for actinide elements, such as monazite, thorium double-phosphates and apatite. Apatite, for example, britholite  $([Ca^{2+}, (PO_4)^{3^-}] - [RE^{3+}, (SiO_4)^4])$  and specifically  $Ca_2Nd_8(SiO_4)_6O_2)^{22}$ , and fluroapatite  $Ca_{10-x}RE_x(SiO_4)_x(PO_4)_{6-x}F_2^{23}$  (where RE is a trivalent rare earth, Nd, La, Gd, etc.) have been studied as potential waste forms particularly for actinides <sup>24</sup>. This interest is primarily because apatites that have retained U and some fission products have been found in the Oklo natural reactor<sup>25</sup>. Apatite has extensive solid solubility for trivalent Pu and Gd (up to ~ 50 wt. %) but the solubility of U is less. Monazite (e.g., CePO\_4) has extensive solid solubility for Pu, U, and Gd, the latter has interest as a neutron absorber for criticality control, and has been extensively studied as a waste form due to its well-characterized mineralogy and its durability in geological conditions<sup>26</sup>. However, it too lacks the lattice sites to incorporate many waste and impurity ions. It is also amorphises due to fission fragment damage, but its durability remains high <sup>27</sup>. The very durable thorium double phosphates (Th<sub>4</sub>(PO<sub>4</sub>)<sub>4</sub>P<sub>2</sub>O<sub>7</sub>) studied by Dacheux et al. <sup>28</sup>, can take

up extensive amounts of Pu and U but their capacity to incorporate Gd and Hf (Hf is a neutron absorber) and their response to alpha-recoil damage do not appear to have been studied. Their non-cubic structure could render them vulnerable to microcracking after radiation damage. The material also does not have sufficient lattice sites to incorporate many of the impurities found in waste streams. Therefore, out of the phosphates, NZP and NTP appear to offer the best alternatives for immobilization of the waste considered here, as they immobilize both Na and P plus have 3 sites for immobilizing fission product ions.

Molybdenum is multivalent and hence its partitioning into the host phases depends upon the processing conditions. If the waste form monolith is consolidated in air Mo will tend to be hexavalent ( $Mo^{6+}$ , ionic radius (IR) = 0.059 nm, VI-fold coordinated); in neutral atmospheres, e.g., Ar, it could exist in the tetravalent form ( $Mo^{4+}$ , IR = 0.063 nm (VI)). Therefore, Mo as  $Mo^{4+}$  should be able to be substituted in the  $Ti^{4+}$  site (IR(VI) = 0.0605 nm) of titanate synroc phases such as rutile, zirconolite, perovskite and pyrochlore. Under reducing conditions it can be reduced to metal. Hence in synroc-C, which is made under reducing conditions Mo forms a metallic alloy phase with other waste stream elements such as Fe, Pd, Ru, Ag, Rh, and Cr<sup>29</sup>.

As part of the Plutonium Immobilization Project the incorporation of ~ 15 wt% Mo and W in the pyrochlore ceramic was studied <sup>30</sup>. The samples sintered in Ar consisted of pyrochlore (Ca<sub>1.03</sub>Gd<sub>0.21</sub>Hf<sub>0.25</sub>U<sub>0.40</sub>Ce<sub>0.20</sub>Ti<sub>1.75</sub>Mo<sub>0.04</sub>W<sub>0.15</sub>O<sub>7</sub>) with approximately 20 vol% of brannerite of composition (Ca<sub>0.09</sub>Gd<sub>0.16</sub>Hf<sub>0.17</sub>U<sub>0.45</sub>Ce<sub>0.22</sub>Ti<sub>1.98</sub>Mo<sub>0.007</sub>W<sub>0.001</sub>O<sub>6</sub>), ~ 5 vol% rutile (Ca<sub>0.003</sub>Hf<sub>0.14</sub>U<sub>0.005</sub>Ti<sub>0.85</sub>O<sub>2</sub>) and ~ 5–10 vol% of scheelite/powellite (Ca<sub>1.02</sub>Mo<sub>0.65</sub>W<sub>0.35</sub>O<sub>4</sub>) solid solution. Similar results were found for the samples sintered in air. Hence, only trace amounts of Mo and W were incorporated into the brannerite or rutile. About 3 times as much W as Mo (on a molar basis) entered the pyrochlore. Consequently, the Ca(W,Mo)O<sub>4</sub> contained about twice as much Mo as W. In a 3.5% H<sub>2</sub> in Ar mixture, the Mo and W were reduced to the metallic state.

Attempts were also made to produce single phase Mo-doped pyrochlore<sup>31</sup>. If Ca was present, e.g., in the CaGdTi<sub>1.5</sub>Mo<sub>0.5</sub>O<sub>7</sub> batch, it was found that the Mo preferentially formed powellite (CaMoO<sub>4</sub>) even under Ar (pO<sub>2</sub> =  $10^{-5}$ - $10^{-6+}$  atm.). The results are summarized in Table 1. Mo samples were made as:

- Mo targeted valence of 6+ by firing in air (CaGdTi<sub>1.5</sub>Mo<sub>0.5</sub>O<sub>7</sub>, Gd<sub>2</sub>TiMg<sub>0.5</sub>Mo<sub>0.5</sub>O<sub>7</sub> and Gd<sub>2</sub>Ti<sub>0.8</sub>Al<sub>0.8</sub>Mo<sub>0.4</sub>O<sub>7</sub>)
- Mo targeted valence of 4+ by firing in Ar (Gd<sub>2</sub>Ti<sub>1.5</sub>Mo<sub>0.5</sub>O<sub>7</sub>).
- Out of interest some samples of the composition Gd<sub>2</sub>Ti<sub>1.5</sub>Mo<sub>0.5</sub>O<sub>7</sub> were also fired in air.

Single-phase Mo-doped pyrochlore was not produced The Mo appears to have a solid solubility limit in the Tipyrochlore of < 0.1 formula units. It is interesting to note that in the W-Mo coupled impurity-doping work the amount of Mo found in samples of the "nominally" 6+ batches sintered at 1350°C for 4 hours in Ar or air <sup>30</sup> was:

- Pu/U-doped 0.04 and 0.02 for Ar and air sinters respectively.
- Ce/U-doped 0.06 and 0.04 for Ar and air sinters respectively.

The phases discussed above were utilized as the basis for potential multi-phase ceramic formulations. The strategy approached is outlined in figure 1. The target phases are based upon the main synroc phases (perovskite, zirconolite, rutile and hollandite) plus pyrochlore into which  $Mo^{4+}$  is substituted for  $Ti^{4+}$ . This will entail processing under controlled atmospheres. For processing in air ( $Mo^{6+}$ ) powellite, monazite, and sodium zirconium and titanium phosphates were considered as viable phases. Key goals were to achieve dense waste forms, with very high waste loadings.

### EXPERIMENTAL

Batches were blended from laboratory grade chemicals. A batch of non-radioactive simulated PUREX type waste (Table 2)  $^{4,6}$  was used to add the fission products. This was supplemented by nitrate solutions where a deficiency was noted relative to the waste streams  $^{2,3}$ .

The Al was added as a fumed alumina (Degussa Aluminium Oxid C), Gd as  $Gd_2O_3$ , Mo which was added as  $MoO_3$  and Na, Ca and additional Ba as hydroxides. Phosphorus was added as phosphoric acid, the Zr as the alkoxide, zirconium tert-butoxide, and titanium as nano-sized Degussa P25 TiO<sub>2</sub>. The rest of the process and impurity ions in addition to those in the bulk simulated PUREX solution, were added as nitrates. The oxides and hydroxides and the zirconium alkoxide were blended together with water to form an aqueous slurry. The nitrates and simulated PUREX waste were then added, followed by the phosphoric acid. The slurry mixture was mixed with a high-speed shear mixer for 10 minutes. The slurry was dried on a hot-plate and then calcined in air at 700°C for 2 hours. The

resulting calcine was then broken up in a mortar and pestle and passed through a sieve (~ 500  $\mu$ m). This material was then wet (water) ball milled for 16 hours, using 10 mm diameter zirconia media in a rubber jar. The milled slurry was dried and the cake formed was broken up with a mortar and pestle. Pellets of ~ 0.5 g were pressed from the powder in 10 mm diameter hardened steel dies at ~ 100 MPa. Samples were then sintered in oxidizing (air) and neutral (high purity Ar, pO<sub>2</sub> = 10<sup>-5</sup> - 10<sup>-6</sup> atm.) atmospheres at temperatures from 900-1100°C.

Samples were examined by theta/2 theta x-ray diffraction (XRD), using a Siemens D500 instrument with Co Kalpha radiation. Scanning electron microscopy (SEM) to take backscattered electron (BSE) and secondary electron (SEI) images was carried out using JEOL JSM6400 and JSM6300 units fitted with Noran energy dispersive x-ray spectroscopy (EDS) systems for chemical analysis. Estimated errors for the EDS measurement are  $\sim \pm 0.02 - 0.03$ formula units. Phase abundances quoted in this work have been estimated by visual estimation diagrams<sup>32</sup>.

Nominal Composition	Prep.	Sinter Temp.	Results
of the major phase "	Koute	(°C)/	
		atm./ Time (b)	
Ma daned pyrachlare		Time (II)	
CaGdTi, Mos Or	ovide	1/150/air//18	XRD nattern reveals pyrochlore nowellite
Mo targeted valency of $6+$	UXIUC	1450/all/40	$(C_2M_0\Omega_1)$ and perovskite
"	"	1/100/air/18	(CalvioO <sub>4</sub> ) and perovskite VPD pattern reveals purechlore, powellite
		1400/a11/40	(CoMoQ) and percuskite
	"	1450/4/9	(CalvioO <sub>4</sub> ) and perovskite
$Gd_2 \Pi_{1.5} WO_{0.5} O_7$		1450/AI/8	A mixture of $Gd_2 \Pi_{0.9} MO_{1.1} O_7$ and $G \downarrow T$
No targeted valency of 4+	1.	1450/4/50	$Gd_2 I1_{1.9}MO_{0.1}O_7.$
22	oxide	1450/Ar/50	As above.
	oxide	1450/air/50	Not single-phase pyrochlore - by XRD. XRD
			pattern is different to the sample sintered in
			Ar (see above)
	alkoxide	1450/air/16	This was composed of $Gd_2Ti_2O_7$ pyrochlore
			with a trace of Mo $(0.02 \text{ formula units})$ and a
			phase of composition Gd <sub>1.7</sub> Ti <sub>0.9</sub> Mo <sub>0.9</sub> O <sub>7 -x</sub>
$Gd_2TiMg_{0.5}Mo_{0.5}O_7$	alkoxide	1450/air/16	Composed of MgMoO <sub>4</sub> ,
Mo targeted valency of 6+			$Mg_{0.25}Gd_{1.79}Mo_{0.31}Ti_{1.58}O_7$ , and
			$Gd_{1.80}Mo_{0.98}Ti_{0.18}O_6$ ,
	alkoxide	1450/air/50	X-ray is similar to the 16 hour sinter
Gd <sub>2</sub> Ti <sub>0.8</sub> Al <sub>0.8</sub> Mo <sub>0.4</sub> O <sub>7</sub>	alkoxide	1450/air/16	Appears to consist of two phases, one of
Mo targeted valency of 6+			which is a pyrochlore with $\sim 0.09$ formula
- •			units of Mo, the other is unidentified at this
			stage but appears to be a Mo-rich phase.
««	"	1450/air/50	XRD pattern is similar to the above
# All these compositions w	vere made with	1 wt. % excess TiO <sub>2</sub> .	

Table I. Summary of the Results for the Mo-doped Pyrochlore Compositions Made

Table II.	Simulated HLW	Used as a Source	of Simulated Fission	Products in this Work
-----------	---------------	------------------	----------------------	-----------------------

Element	Waste	Waste	Moles	Moles ions	Element	Waste	Waste ions	Moles	Moles ions
	oxide basis	ions (wt.		(%)		oxide basis	(wt. %)		(%)
	(wt. %)	%)				(wt. %)			
Ag	0.20	0.25	1.85E-03	0.25	Ni	0.25	0.31	4.26E-03	0.58
Ba	3.59	4.48	2.61E-02	3.54	Р	0.73	0.91	2.36E-02	3.19
Cd	0.18	0.22	1.60E-03	0.22	Pd	3.30	4.12	3.10E-02	4.20
Ce	12.75	15.91	9.10E-02	12.32	Rh	1.03	1.29	1.00E-02	1.35
Cr	0.58	0.72	1.12E-02	1.51	Ru	5.84	7.29	5.78E-02	7.82
Cs	7.94	9.91	5.97E-02	8.09	Sr	2.31	2.88	2.63E-02	3.57
Fe	2.73	3.41	4.89E-02	6.62	Те	1.52	1.90	1.19E-02	1.61
Gd	3.29	4.11	2.09E-02	2.83	Y	2.01	2.51	2.26E-02	3.06
Mo	8.87	11.07	9.25E-02	12.52	Zr	9.44	11.78	1.03E-01	14.01
Nd	13.56	16.93	9.40E-02	12.73					
					TOTAL	80.12	100.00	0.74	100.00

Element	1 2 3		3	4 5		
	NZP - powellite	pyrochlore-NTP- perovskite	NTP-perovskite- powellite	NZP-powellite	NTP-powellite	
	(Mole %)	(Mole %)	(Mole %)	(Mole %)		
Additives			. ,			
Ca	9.0E+00		2.4E+01	2.2E+01	2.2E+01	
Zr	1.1E+01			1.6E+01		
Gd		3.0E+01	4.0E+00			
Ti		1.1E+01	2.1E+01		1.6E+01	
Р				1.3E+01	1.3E+01	
Additive	20.5	41.4	48.6	50.7	51.1	
s Total						
%						
Waste						
Ag	2.4E-03	1.7E-03	1.5E-03	6.0E-03	5.3E-03	
Al	4.0E+00	3.0E+00	2.6E+00	1.3E+00	1.3E+00	
Ba	1.7E-01	1.2E-01	1.1E-01	8.5E-02	7.5E-02	
Cd	2.0E-03	1.5E-03	1.3E-03	5.2E-03	4.6E-03	
Ce	1.2E-01	8.5E-02	7.4E-02	2.9E-01	2.6E-01	
Cr	3.3E-01	2.4E-01	2.1E-01	2.0E-01	1.9E-01	
Cs	7.6E-02	5.6E-02	4.9E-02	1.9E-01	1.7E-01	
Fe	1.5E+00	1.1E+00	9.6E-01	9.1E-01	8.9E-01	
Gd	2.7E-02	3.9E-02	8.1E-02	6.8E-02	6.0E-02	
Mg	1.5E+00	1.1E+00	9.5E-01	9.2E-01	9.3E-01	
Mo	3.5E+01	2.6E+01	2.3E+01	2.2E+01	2.2E+01	
Na	1.3E+01	9.7E+00	8.5E+00	8.2E+00	8.2E+00	
Nd	1.2E-01	8.7E-02	7.7E-02	3.0E-01	2.7E-01	
Ni	1.6E-01	1.2E-01	1.0E-01	1.4E-02	1.2E-02	
Р	2.2E+01	1.6E+01	1.4E+01	1.4E+01	1.4E+01	
Pd	3.9E-02	2.9E-02	2.5E-02	1.0E-01	8.9E-02	
Rh	1.3E-02	9.3E-03	8.2E-03	3.2E-02	2.9E-02	
Ru	7.3E-02	5.4E-02	4.7E-02	1.9E-01	1.6E-01	
Sr	2.0E-01	1.4E-01	1.2E-01	8.5E-02	7.5E-02	
Те	1.5E-02	1.1E-02	9.7E-03	3.9E-02	3.4E-02	
Y	2.9E-02	2.1E-02	1.8E-02	7.3E-02	6.5E-02	
Zr	8.3E-01	7.0E-01	6.2E-01	6.9E-01	3.0E-01	
Waste	79.5	58.6	51.4	49.33	48.80	
Total %						
Total %	100	100	100	100	100	

Table III. Composition, Cation-basis, for Target Compositions Tested

### **RESULTS AND DISCUSSION**

# Target Composition 1 - NZP with Powellite

This composition (Table 3) was made deficient in Zr and P relative to that needed to make NZP. The aim was to test if the Mo would enter the P and/or Zr lattice sites and what phases would form. This formulation would give the highest waste loading, ~ 80 mole % (77 wt%). NZP has the general structure  $[M_1][M_2][Zr^{VI}_2][[P^{IV}_2]O_{12}$  and is a "3-dimensional hexagonal framework of PO<sub>4</sub> tetrahedra sharing corners with ZrO<sub>6</sub> octahedra". The large Na ions occupy two sites, one in the columns of Zr octahedra [M'] and one in the between the ribbons of the Zr octahedra [M"]. The former is usually occupied in preference to the latter. These multiple lattice sites of variable size make NZP a potentially good waste form as it will be able to incorporate large, medium and small fission product, process and impurity ions found in real waste streams. In this work the target is to place the large ions Ag, Ba, Ca, Sr, Cs, Cd, and the bigger rare earth (RE) elements in the Na site. The RE, platinoids, Transition Metals, Mg and possibly Y are aimed to occupy the Zr site. The P site would be occupied by P and possibly Al. An earlier paper claimed

that Mo and even Te could occupy the P site. The reported Mo<sup>+6</sup> size (ionic radius (IR)(IV) = 0.41Å) is much larger than reported for P<sup>+5</sup> (IR(IV) = 0.17Å)<sup>33</sup> so significant substitution is unlikely. Some Ca was added to "mop-up" the residual Mo as powellite - CaMoO<sub>4</sub>.

A sample was sintered for 16 hours at 1100°C in air (figure 2). X-ray diffraction data indicated that the main phases were powellite and NZP plus additional unidentified peaks. In the SEM (figure 2) the following phases were identified.

1.  $\sim$  50-60 vol% NZP of variable measured composition. No significant amounts of other waste ions were detected in this phase and Mo was not detected in the EDS analysis.

2.  $\sim 10 - 20$  vol% Powellite - (Ca,RE,Na)MoO<sub>4</sub> phase (RE = rare earth element, in this case Gd, Nd and Ce) with variable composition also containing also containing some Sr and Ba.

3. ~ 10 - 20 vol% of a Na-Mg-Fe-Mo-O phase ~ composition (NaMgFeMo<sub>3</sub>Mo<sub>3</sub>O<sub>12</sub>) with traces of Nd, Zr, P, Al, and possibly Ba.

- 4. ~ 3-7 vol% AlPO<sub>4</sub> (with Na, Zr and Fe detected)
- 5. a small amount (< 1 vol%) of (Ru,Rh,Pd,Cr,Mo,P) $O_x$

The Mo<sup>+6</sup> or Al<sup>3+</sup> (IR(IV) = 0.39Å) did not significantly substitute in P site in NZP, something already suspected from ionic sizes. The most likely route for Mo incorporation in NZP appears to be as Mo<sup>4+</sup> in the Zr site of NZP or the Ti site of NTP. The formation of a Re-Na molybdate is possibly undesirable as Na-molybdate has a low aqueous durability; although this would need to be tested and the material has the powellite structure. However, it should be noted that scheelite structured SrMoO<sub>4</sub> was proposed as a phase in supercalcine waste forms. It was claimed that unlike alkali molybdate (A<sub>2</sub>MoO<sub>4</sub>, with A = Na, Cs and Rb), alkali-earth molybdates (Sr, Ca and Ba) had "reasonable leaching resistances" <sup>34</sup>. The structure of Na-Mg-Fe-Mo-O phase is unknown at this stage.

The 1100°C Ar-sintered sample melted. In the sample sintered in Ar at 1000°C for 16 hours, the following phases were observed:

- 1. ~40-50 vol% of powellite, (Na,Gd,Ca)MoO<sub>4</sub> with a small amount of other REs and Sr.
- 2. ~ 30-40 vol% of NZP, with significant Mo in the material and traces of Al, Ca, Fe, REs, and Ru.
- 3. ~ 3-7 vol% Ba(Mo,Zr) $O_{3+x}$  phase that contains Ru, Na, and REs
- 4. ~ 3-7 vol% AlPO<sub>4</sub> (with Na, Mo, Zr and Fe detected).
- 5. <1 % ZrO<sub>2</sub> and possibly  $MoO_2$

These phases are similar to the sample sintered in air indicating that the composition may be reasonably stable under variable processing conditions. However, aqueous durability of the Ba-molybdate phase would need to be confirmed.

The fact that the composition formed NZP and powellite as the main phases is promising as is the high waste loading achieved. In addition, the fact that these phases remained the dominant phases despite changes in the processing atmosphere is also promising as it implies a broad processing window. The sample was not fully homogeneous and the porosity was estimated to be ~ 10-15 vol%. It is probable that some segregation has occurred on drying. This has been observed previously by the author in tray dried synroc-C materials. While not explicitly part of this work clearly some development of the processing route would be required for these materials to improve homogeneity and optimize the sintering schedule. The formation of secondary phases means reformulation with a reduction in the waste loading from the 77 wt% is necessary. The addition of more Ca to capture the Mo<sup>6+</sup> in powellite, and Zr and P to capture the Na in NZP is recommended.

#### Target Composition 2 - Pyrochlore-NTP-Perovskite system

The strategy with this composition (Table 3) was to incorporate Mo in the +4 state into pyrochlore and perovskite phases with NTP to incorporate the fission products and rare earths. This would be achieved by the addition of  $Gd^{3+}$  to couple with Na<sup>+</sup> and substitute into perovskite A-site (ABO<sub>3</sub>), with B as Mo<sup>4+</sup> or Ti<sup>4+</sup>. Additional RE<sup>3+</sup> was added to promote pyrochlore formation with Mo<sup>4+</sup> and Ti<sup>4+</sup>. Additional Ti would combine with residual P and Na to form



Fig. 1. Potential host phases for the incorporation of Mo, P and Na-rich waste streams.



Fig. 2. SEM micrograph BSE image of the target composition - NZP with powellite, waste loading ~ 77 wt%, sintered in air at 1100°C for 16 hours. The light phase is powellite, other phases detected were NZP (Z), an unidentified phase of ~ composition NaMgFeMo<sub>3</sub>Mo<sub>3</sub>O<sub>12</sub> (X), aluminium phosphate (A) and platinoid oxide (small < 1 $\mu$ m white dots). Porosity ~ 10 vol% is also present.

NTP. Tetravalent Mo would also partially substitute into the Ti site (Table 1). In this batch the waste loading was  $\sim$  40 wt%. This batch was sintered in air and Ar.

In the sample sintered in air at  $1100^{\circ}$ C for 16 hours the NTP phase was not detected. XRD patterns analyzed as GdPO<sub>4</sub>, powellite and rutile. The following phases were detected by SEM:

- ~ 60-70 vol% powellite (Na, Mg, Gd)MoO<sub>4</sub>) of variable composition, and also contained Ti, Ce, Nd, Ru and Pd. It also contained some Ca, which may have come from cross-contamination. The XRD indicates that this phase has a powellite structure.
- 2. ~ 20-30 vol% monazite, GdPO<sub>4</sub> that contains some Mg, Al, Zr, Ce, and Nd.
- 3. ~ 5 vol% rutile with Mo, Ni, Al, Zr, Fe, Cr, Ru, Rh, Pd and Gd.
- 4. A small amount (~ 1-2 vol%) of a Mg-Mo-O phase which contains traces of Na, Ba, Fe, Ni, Al, Zr, Ti and Gd.

With Mo in the 6+ state it appears that the rare earth co-substitutes with the Na to form a (Na,Mg,Gd)MoO<sub>4</sub> phase with the powellite/scheelite structure.

After sintering in Ar for 16 hours at 1000°C (figure 3) the phases detected were:

- 1. ~ 25-35 vol% GdPO<sub>4</sub>, monazite, which has traces of Zr, Al, Mo, Sr, Mg and Ba.
- 2. ~ 20-30 vol% powellite, (Na,Gd)MoO<sub>4</sub> and traces of Nd, Ce, Zr, Al, Fe, Ni and Ru.
- 3. ~ 3-7 vol% of Mo-Ti oxide approximate composition  $Ti_{0.78}Mo_{0.09}Zr_{0.02}Al_{0.03}Fe_{0.03}Cr_{0.02}Gd_{0.01}O_2$ .
- 4. ~2-5 vol % spinel mainly composed of Mg, Ni, Fe, Cr and Al with traces of Mo, Zr, Ti and Gd.
- 5. ~ 15-25 vol% of a Gd-Mo-rich phase that from XRD data appears to be a Gd-Mo-Ti pyrochlore, which contains some Al, Ce, Nd, Mg, Ba, Sr and Na.



Fig. 3. SEM micrograph BSE image of the target Composition - pyrochlore-NTP-perovskite, waste loading ~ 40 wt%, sintered in Ar at 1000°C for 16 hours. The phases detected were powellite (P), monazite (M, light phase), pyrochlore (Y), Mo-Ti oxide (O, dark grey) and a small amount of spinel (S, black). Porosity ~ 10-15 vol% is also present.



Fig. 4. SEM micrograph BSE image of the target Composition – NZP-Powellite, waste loading ~ 53 wt%, sintered in air at 1100°C for 16 hours. The phases detected were powellite (P), NZP (Z), a Ca-Na-Mg-phosphate (C), and Zr-oxide (O). Porosity ~ 5-10 vol% is also present.

After sintering at 1100°C in Ar the sample appears to be similar to the 1000°C sample. Phases detected by XRD were powellite, monazite and pyrochlore with an unidentified peak at ~ 3.33 Å. The pyrochlore peak is much more intense than in the 1000°C sinter. EDS data from the SEM results suggests that Gd, Na, Ba and Sr are occupying the A-site of the pyrochlore with Mo, Ti, Al and Mg the main ions on the B-site. There is measured excess of B site ions, suggesting that there are vacancies in the A-site. Given that the pO<sub>2</sub> in a tube furnace when using high purity bottled argon is likely to be  $10^{-5}$  to  $10^{-6}$  atm., then it is possible that the Mo has not been fully reduced to Mo<sup>4+</sup> and may still be present in the hexavalent state. Hexavalent Mo is has been found in pyrochlore <sup>35</sup>. More experimental work needs to be done to clarify the situation.

#### Target Composition 3 - NTP-perovskite-powellite

The strategy with this composition (Table 3) was to use trivalent RE ions, in this case Gd to co-substitute with Na for Ca to form a titanate perovskite. Ca was added to promote powellite formation. It was envisaged that additional Ti would combine with residual P and Na to form NTP. In this batch the waste loading was ~ 55 wt%.

In the sample sintered in air at 1100°C for 16 hours the following phases were detected.

- 1. 40-50 vol% of a Ca-Mg-Gd-Na-Ti-phosphate phase that also contained Mo, Zr, Ce, Nd, Ba, Sr, transition metals and platinoid elements. This phase was not NTP by XRD.
- 2. 45-55 vol% of powellite  $CaMoO_4$  with small amounts of Na, Mg, the rare earths plus traces of the other fission products.
- 3.  $\sim$  2-3 vol% of a Rare Earth Al phosphate, which contains some Mg.

The sample sintered in Ar at 1000°C appeared to be incompletely reacted as it contained several minor phases. It was composed of:

- 1. 30-40 vol% of a Ca-Mg-Gd-Na-Ti-phosphate that also contained Mo, Zr, Ce, Nd, Ba, Sr, transition metals and platinoid elements.
- 2. 30-40 vol% of powellite CaMoO<sub>4</sub> with significant amounts of Na and the rare earths plus traces of Ba, Sr and Fe.
- 3. ~5-7 vol% of monazite.
- 4. ~ 1 vol% of Ti-oxide, which contains small amounts of Mo, Zr, Ca, Fe, and Cr
- 5.  $\sim 2-5$  vol% of a Na-Mg-Al-Ti-O phase.
- 6. ~ 2-5 vol% of a (Na,Ca,RE)(Ti,Al,P)-oxide phase.
- 7.  $\sim$  2-5 vol% of a Mg-Al spinel.
- 8. ~1-3 vol% of NZP (Na,Ca, RE)(Zr,Mo,Ti,Ru)<sub>2</sub>(P,Al)<sub>3</sub>O<sub>12</sub>

Clearly this formulation did not achieve its target. It appears that Na and Gd prefer to for powellite or phosphates that the Na-RE-perovskite. Hence, it is concluded that perovskite is not a viable target phase for this waste stream.

### Target Composition 4 - NZP-powellite - Mo as 6+

In this batch the target phases were powellite and NZP (Table 3). The waste loading was ~ 53 wt%. Samples were sintered in air at 900, 1000 and  $1100^{\circ}$ C.

The sample sintered in air at 1100°C for 16 hours (figure 4) consisted of :

- 1. 50-60 vol% powellite, with traces of Na, RE and P.
- 2. 30-40 vol% NZP that appears to be NZP from the XRD analysis, but was analyzed by EDS as ~  $Na_2Zr_2P_3O_{12}$ .
- 3. ~ 5-10 vol% Ca-phosphate (~ (Ca,Na,Mg,RE)<sub>3</sub> $P_2O_8$ ) with traces of Al, Zr, Cr and Fe.
- 4. ~ 2-3 vol% ZrO<sub>2</sub>.

The sample sintered at 1000°C in air is similar except that the grain size is smaller,  $5-15\mu m$ , compared to  $10-25\mu m$  for the 1100°C sintered sample. There is little difference in the x-ray patterns between the samples sintered for 16 hours at 900, 1000 and 1100°C in air.

In Ar the same phases were observed except instead of zirconia  $MoO_2$  formed. There are also small changes in the chemistry of the phases. The Ca-phosphate for instance contains Mo when sintered in Ar. The NZP composition is close to  $NaZr_2P_3O_{12}$  with traces of other batch elements.

### Target Composition 5 - NTP-Powellite - Mo as 6+

In this batch the target phases were powellite and NTP (Table 3). The waste loading was ~ 59 wt%. Samples were sintered in air at 900, 1000 and  $1100^{\circ}$ C.

The sample sintered in air at 1100°C for 16 hours was composed of:

- 1. ~ 25-30 vol% NaTi<sub>2</sub>P<sub>3</sub>O<sub>12</sub>, which contains Al, Fe, Gd and possibly a trace amount of Cs.
- 2. ~30-40 vol% powellite, which contains traces of Na, RE, Ba, Sr, Fe, Ti and Al.
- 3. ~25-35 vol % of a Ca-phosphate, ~(Ca,Na,Mg,RE)<sub>3</sub> $P_2O_8$ , with traces of transition metals, Al and Ti.
- 4. ~ 10-20 vol% rutile.
- 5. < 1 vol% of PdO<sub>2</sub> precipitates associated with the rutile as a small core in the rutile grain.

The sample sintered at 1000°C were similar but with a smaller grain size.

In a sample sintered in Ar at 1000°C for 16 hours the same phases formed as the air sinter. However, some molybdenum oxide forms and Mo was detected in the rutile.

The results are promising with viable waste form phases present in all the formulations tested. The next step would be to refine the composition and a target system of NZP-monazite-powellite appears to be a reasonable system in which to obtain dense ceramic waste forms with high waste loadings. At this stage the processing route has not been refined. The total porosity of the samples was typically between 5 and ~ 20 %. With optimization of the sintering schedule and powder processing dense ceramic waste forms should be possible, especially if consolidation routes such as hot-isostatic pressing are used. Additional work not reported here has been carried out on fabricating end members of the phases tested here and will be reported elsewhere. Another issue being addressed is the fate of Cs, which can be volatile in oxidizing conditions and may also form soluble molybdate phases. Cs-doped end-members have been made and are being examined.

From a processing viewpoint the fact that samples were able to be sintered in air to make viable compositions is favorable as sintering without atmosphere control complexities would render processing much easier. However, if a neutral atmosphere is used, such as that found in a hot-isostatic pressing can the compositions are still viable<sup>36</sup>. The fact that materials with viable waste form phases, such as pyrochlore, can also be formed in neutral atmosphere means that alternative processes to sintering such as hot-isostatic pressing may also be a suitable production route for these waste forms.

# CONCLUSIONS

The five compositions tested are promising. The waste loading ranged from 40-77 wt%. Some refinement is required to optimize the composition, then testing of the durability will be required. The development of a waste form with a high waste loading based upon the phosphates NZP and monazite, coupled with the use of the powellite phase to incorporate the Mo and some Na is viable. However, the durability of the powellite phase needs to be verified.

# ACKNOWLEDGMENTS

Thanks to Mugdha Bhati for her help with sample preparation and SEM. Thanks also to Hujing Li for his help with the SEM analysis.

### REFERENCES

- <sup>2</sup> R. Do Quang, V. Petitjean, F. Hollebecque, O. Pinet, T. Flament, A. Prod'homme, "Vitrification of HLW Produced by Uranium/Molybdenum Fuel Reprocessing in COGEMA's Cold Crucible Melter", CD-ROM, *ICEM'03 - 9th International Conference on Environmental Remediation and Radioactive Waste Management*, September 21-25, 2003, Oxford, England, ASME, Fairfield, NJ, USA, 2003.
- <sup>3</sup> R. Do Quang, V. Petitjean, F. Hollebecque, O. Pinet, T. Flament and A. Prod'homme, "Vitrification of HLW Produced by Uranium/Molybdenum Fuel Reprocessing in Cogema's Cold Crucible Melter", WM'03, CD-ROM, Feb 23-27, Tucson, AZ, 2003.
- <sup>4</sup> E.R. Vance, M.W.A. Stewart and G.R. Lumpkin, "Immobilisation of Sodium and Potassium in Synroc", J. Mater. Sci., 26 2694-2700 (1991).
- <sup>5</sup> H. Mitamura, S. Matsumoto, M.W.A. Stewart, T. Tsuboi, M. Hashimoto, E.R. Vance, K.P. Hart, Y. Togashi, H. Kanazawa, C.J. Ball and T.J. White, "α-Decay Damage Effects in Curium-Doped Titanate Ceramic Containing Sodium-Free High Level Nuclear Waste", J. Am. Ceram. Soc, **77** [9] 2255-2264 (1994).
- <sup>6</sup> M.W.A. Stewart and W.J. Buykx, "Hot-Pressing Behaviour of Synroc Containing Sodium-Bearing Nuclear Wastes", p. 711-716 in Austceram 90 - *Ceramics Technology - sharing the knowledge*, ed. P.J. Darragh and R.J. Stead, Proceedings Austceram 90, Perth W.A., Trans Tech Publications, Switzerland, Key Engineering Materials, **48-50**, 711-716 (1990).

<sup>&</sup>lt;sup>1</sup> O. Bukharin, "Making fuel less tempting", Bull. Atomic Scientists, July/August pp. 44-49 (vol. 58, no. 04) 2002.

- <sup>7</sup> E.R. Vance, P.J. Angel, D.J. Cassidy, M.W.A. Stewart, M.G. Blackford and P.A. McGlinn, "Freudenbergite: A Possible Synroc Phase for Sodium-Bearing High Level Waste", Journal of the American Ceramic Society, **77** [9] 2255-2264 (1994).
- <sup>8</sup> R. Roy, E. R. Vance and J. Alamo, "[NZP], A New Radiophase for Ceramic Nuclear Waste Forms," *Mater. Res. Bull.* 17:585-589 (1982).
- <sup>9</sup> Li J. Yang, S. Komarneni and R. Roy, "Titanium-Phosphate (NTP) Waste Form," *Advances in Ceramics* 8:255-262. Ed. G.G. Wicks and W.A. Ross, *American Ceramic Society*, Columbus, OH (1984).
- <sup>10</sup> B.E. Scheetz, D. K. Agrawal, E. Breval and R. Roy, "Sodium Zirconium Phosphate (NZP) as a Host Structure for Nuclear Waste Immobilization: A Review," *Waste Management* 14(6):489-505 (1994).
- <sup>11</sup> V.I. Petkov and A.I. Orlova, "Phase analysis and crystallographic properties of the phosphate systems MO-ZrO<sub>2</sub>-P<sub>2</sub>O<sub>5</sub> (M = Mg, Ca, Sr, Ba)", J. Thermal Analysis, **54** 71-77 (1998).
- <sup>12</sup> D-M. Liu, "Thermal conduction behaviour of (CaX)Zr<sub>4</sub>(PO<sub>4</sub>)<sub>6</sub> ceramic", J. Mater. Sci. Let., **13**, 129-130, (1994).
- <sup>13</sup> A.I. Orlova, V.I. Petkov, O.V. Egorkova, V.S. Kurazhkovskaya, D.V. Kemenov, O.V Skiba, "Elaboration of phosphate ceramics as a safe form for halide salt waste immobilization of 'dry' fuel reprocessing", pp. 1253-1258 in Proceedings. *Global '97: International conference on future nuclear systems*, Tokyo (Japan). Atomic Energy Society of Japan, Yokohama (Japan). 5-10 Oct, 1997.
- <sup>14</sup> V.I. Pet'Kov, M.V. Surhanov, "Immobilisation of molybdenum from fuel reprocessing wastes into sodium zirconium phosphate ceramics", Czechoslovakian. J. Phys., **53** A671-A677, Suppl. A, (2003).
- <sup>15</sup> A.I. Orlova, "Isomorphism in d- and f-element phosphates having framework crystal structure and crystallochemical conception of NZP matrix for radionuclide immobilization", Czechoslovakian J. Phys., **53** A649-A655, Suppl. A, (2003).
- <sup>16</sup> V.I. Pet'kov, A.I. Orlova, I.G. Trubach, T. Demarin, V.S. Kurazhkovskaya, "The immobilisation of nuclear waste materials containing different alkali elements into single-phase NZP based ceramics" Czechoslovakian J. Phys., **53** A639-A648, Suppl. A, (2003).
- <sup>17</sup> H.T. Hawkins, Scheetz, B.E., and Guthrie, G.D., Jr. Preparation of Monophasic [NZP] Radiophases: Potential Host Matrices for the Immobilization of Reprocessed Commercial High-level Wastes. pp. 387-394. in *Scientific Basis for Nuclear Waste Management XX*, Materials Research Society Symp. Proc. **465**, Gray, W. J. and Triay, I. R., eds., (1997)
- <sup>18</sup> V.N. Zyryanov and E.R. Vance, "Comparison of Sodium Zirconium Phosphate-structured HLW Forms and Synroc High-Level Nuclear Waste Immobilisation", pp 341-348 in *Scientific Basis for Nuclear Waste Management XX*, ed. J. Gray and I.R. Triay, Materials Research Society, Warrendale, PA, USA, 1997.
- <sup>19</sup> Y. Hirose, A. Suzuoki, T. Fukasawa, D.K. Agrawal, H.L. Hawkins, B.E. Scheetz, "An alternative conditioning method for the wastes from LWR fuel reprocessing" pp. 1181-1186 in Proceedings. *Global '97: International conference on future nuclear systems*, Tokyo (Japan). Atomic Energy Society of Japan, Yokohama (Japan). 5-10 Oct 1997.
- <sup>20</sup> C.S. Hong, P. Ravindranathan, D.K. Agrawal and R. Roy, "Synthesis of NaZr<sub>2</sub>P<sub>3</sub>O<sub>12</sub> powders by evaporative decomposition of solutions, J. Mater. Sci. Let., **13**, 1015-1016, (1994).
- <sup>21</sup> A.I. Orlova, Yu.F. Volkov, R.F. Melkaya, L.Yu. Masterova, I.A. Kulikov, V.A. Alferov, "On the synthesis and radiation stability of the NZP structure phosphates containing f-elements", Radiochemistry, **36**, 322-325, 1995.
- <sup>22</sup> W.J. Weber, R.K. Eby and R.C. Ewing, "Accumulation of structural defects in ion-irradiated Ca<sub>2</sub>Nd<sub>8</sub>(SiO<sub>4</sub>)<sub>6</sub>O<sub>2</sub>", J. Mater. Res, 6 [6], 1334-1345 (1991).
- <sup>23</sup> C. Meis, "Computational Study of the Incorporation and Lattice Migration in Fluroapatites", Extended Abstract, pp. 13-15 in *HLW and Pu Immobilisation Meeting*, Ed. C. Meis, Workshop at CEA-INSTN Saclay, 22-23 April 1999, Saclay, France, DCC/DPE/SPCP-Saclay.
- <sup>24</sup> W.J. Weber, R.C. Ewing and A. Meldrum, "The kinetics of alpha-decay-induced amorphization in zircon and apatite containing weapons-grade plutonium or other actinides", J. Nucl. Mater., **250**, 147-155 (1997).

- <sup>25</sup> I.W. Donald, B.L. Metcalfe and R.N. Taylor, "Review, The immobilization of high level radioactive wastes using ceramics and glasses", J. Mater. Sci. **32**, 5851-5887 (1997).
- <sup>26</sup> L.A. Boatner and B.C. Scales, "Monazite" pp. 495-564 in *Radioactive Waste Forms for the Future*, Ed. W. Lutze and R.C. Ewing, North-Holland, Amsterdam, 1988.
- <sup>27</sup> E.R. Vance and K.K.S. Pillay, "Fission Fragment Damage in Crystalline Phases Possibly Formed in Solidified Radioactive Waste", Rad. Eff., 62, 25-38 (1982).
- <sup>28</sup> N. Dacheux, A. C. Thomas, B. Chassigneux, E. Pichot, V. Brandel and M. Genet, "Study of Th<sub>4</sub>(PO<sub>4</sub>)<sub>4</sub>P<sub>2</sub>O<sub>7</sub> and Solid Solutions with U(IV), Np(IV) and Pu(IV):Synthesis, Characterization, Sintering and leaching Tests", pp. 85-92 in *Scientific Basis for Nuclear Waste Management XXII*, Eds. D. J. Wronkiewicz and J. H. Lee, Materials Research Society, Warrendale, PA, USA, 1999.
- <sup>29</sup> M.L. Carter, M.W.A. Stewart, S.H.F. Leung, M. Colella and E.R. Vance, "Microstructure of Inactive Synroc-C Samples Produced by Different Hot-Consolidation Methods" in *Mass Charge and Transport in Ceramics*, ed. K. Koumoto, L.M. Sheppard and H. Matsubara, Ceram. Trans., Vol. 71, pp 491-504 (1996), Am. Ceram. Soc., Westerville, OH.
- <sup>30</sup> M.W.A. Stewart, E.R. Vance, R.A. Day, S. Leung. A. Brownscombe, M.L. Carter and B.B. Ebbinghaus, "Impurity incorporation in Pyrochlore-rich Ceramics", 101st American Ceramic Society Annual Meeting, Indianapolis, April 25-28, 1999, *Environmental Issues and Waste Management Technologies in the Ceramic and Nuclear Industries V*, ed. G.T. Chandler and X. Feng, Ceram. Trans., Vol. **107**, 569-576 (2000).
- <sup>31</sup> M.W.A. Stewart, E.R. Vance, A. Jostsons, P.A. Walls, K.P. Hart, S. Moricca, R.A. Day, G.R. Lumpkin, C.J. Ball and D.S. Perera, *Final report to Lawrence Livermore National Laboratory for Contract B345772*, ANSTO report R00m031, Oct. 2000.
- <sup>32</sup> R.D Terry and G.V Chilingar, "Summary of `Concerning some additional aids in studying sedimentary formations' by M. S. Shvetsov", J. Sediment. Petrol., Vol. 25, (1955), 229-234.
- <sup>33</sup> R.D. Shannon, "Revised Ionic Radii and Systematic Studies of Interatomic Distances in Halides and Chalcogenides", Acta Cryst., "A32, 751 (1976).
- <sup>34</sup> G.R. McCarthy, "High-level Waste Ceramics: Materials Considerations, Process Simulation and Product Characterization", Nucl. Tech., **32**, Jan., 92-105, (1977).
- <sup>35</sup> R.A. McCauley and F.A. Hummel, "New Pyrochlores of Charge-Coupled Type", J. Solid. State Chem, **33**, 99-105, (1980).
- <sup>36</sup> R.A. Day, B.D. Begg, S. Moricca, M.W. Stewart, R. Muir, E.R. Vance, "Low-Risk Alternative Synrocansto Waste Forms and Process Technologies for Ineel Sodium-Bearing Waste and Calcines", paper 5263, *Waste Management 05 (WM'05)*, CD-ROM, Feb. 27-Mar. 3, 2005, Tucson, AZ, USA.